



Application of Clay Materials for Sorption of Radionuclides from Waste Solutions

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Abstract: The wide application of nuclear resources in various fields has resulted in the production of radioactive waste, which poses a serious threat to lives and the environment. Nuclear waste contains long-lived radionuclides and, due to its mobility in environments, the proper management of generated waste is necessary. To impede the mobility of radionuclides in environments, various materials have been tested as suitable sorbents under different experimental conditions. In this review, we thoroughly discuss some key and recent contributions to the application of natural clays (NCs) and modified/functionalized clays (MCs) for the sorption of various radionuclides in their cationic and anion forms from (simulated) waste solutions under different experimental conditions. More specifically, we discuss the key developments toward the use of natural clays for the efficient sorption of various radioactive contaminates. Later, this review targets the modification/functionalization of natural clays using various organic moieties to improve their removal capacities for various radionuclides/hazardous ions present in waste solutions. Finally, we summarize the major aspects and highlight the key challenges to be addressed in future studies to further enhance the application of clays and clay-based materials for selective and effective removal of various radionuclides from waste solutions.



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Copyright: © 2023 by the authors. Licensee MDPI, Basel, Switzerland. This article is an open access article distributed under the terms and conditions of the Creative Commons Attribution (CC BY) license (https:// creativecommons.org/licenses/by/ 4.0/). **Keywords:** natural clays; bentonite; montmorillonite; functionalized clays; sorption; nuclear waste solution

1. Introduction

The development of nuclear technology in various sectors, including weapons, power, aerospace and medicine, have generated a large amount of extremely hazardous radioactive waste (containing long half-life radionuclides) [1–8]. Besides the aforementioned anthropogenic activities, radionuclides can also be released and entered into the environment due to nuclear accidents/disasters, such as the Chernobyl and Fukushima Daiichi nuclear power plant incidents [9–26]. The radiotoxicity output and exposure of the produced radioactive wastes/radionuclides are a tremendous threat to both humans as well as the wider geosphere/environment [27,28]. Once released, the radionuclides have the tendency to migrate in the environment [29,30]. After the Fukushima nuclear disaster, the radioactive levels of ²³⁹Pu, ²³⁴U and ²³⁸U suddenly increased, particularly in Hawaii, Alaska and California [31,32]. Most of these radionuclides were supposed to be migrated over the Pacific Ocean due to the wind orientation. In order to hinder the mobility of released radionuclides in various geological and environmental media, their efficient removal/sorption from aqueous waste solutions and proper management (solidification, immobilization and underground disposal) using various materials/matrices, are being investigated.

In the last few decades, numerous materials have been studied and tested for the sorption of radionuclides from (simulated) waste solutions under different experimental conditions [33-45]. Among various sorbents, clays are widely studied as natural sorbents for wastewater decontamination and as a potential barrier in landfills to hamper and stop the leaching of radioactive materials/radionuclides into the subsoil and groundwater [46–54]. Clays are a ubiquitous, key component of soil, sediment and bedrock, and are the most abundant and cheapest materials, which has attracted significant interest in the scientific community over the past few decades [55–60]. Fundamentally, clays are hydrous aluminum silicates commonly formed by prolonged chemical weathering of silicate-bearing rocks and are composed of aluminum and silicon ions bonded into tiny, thin plates by interconnecting oxygen and hydroxide ions. The main groups of clays include kaolinite, montmorillonite (bentonite) and illite [61]. The crystal structure of montmorillonite/bentonite is composed of two layers of silica tetrahedra with a central layer of alumina octahedron between them (TOT type). Bentonite clay has been widely applied as a potential buffer material in highlevel radioactive waste (HLW) repositories due to its ability to absorb water molecules between the sheets, resulting in a significant expansion and the different exchangeable cations can replace (hazardous) ions in the structure [62,63].

Clays readily exchange cationic radionuclides in nuclear waste solutions due to the presence of alkali/alkaline earth metals in their crystalline aluminosilicate frameworks and they are known for their sorption of cationic contaminants due to their large surface area, high thermal stability, porous structure and cation exchange capacity (CEC) [64,65]. Additionally, significant developments have been made toward the modification of natural clays (NCs) to improve their affinity and selectivity as potential sorbents for the efficient removal of various contaminants from wastewater [66–69]. The NCs have been functionalized and modified via chemical alteration by incorporating organic moieties (such as quaternary amines and others) to form organoclays, which efficiently enhanced their affinity and sorption capacities for extremely hazardous anionic contaminants present in waste solutions [70,71].

This review presents the key developments in the design and application of various NCs and modified clay (MC) materials used for efficient and selective sorption of various hazardous radioactive cationic and anionic species from (simulated) waste solutions with a major emphasis on the pioneering examples and recent developments. Sections 2 and 3 of this review present the core contents on the application of NCs and MCs respectively, for the removal of various radionuclides/ions under different experimental conditions. In Section 2, we discuss some of the key and recent progress on the application of NCs (bentonite/montmorillonite, etc.) for the effective removal of various cationic pollutants. Section 3 encompasses the design and modification of NCs to the organoclays using different organic moieties (in the optimized experimental conditions) and their use as potential sorbents for the efficient removal of hazardous species present in radioactive waste solutions. In Section 4, we offer the concluding remarks and highlight the key concerns that need to be addressed to meet the future challenges of the selection, design and application of suitable clay minerals/materials for the efficient removal of various cationic and anionic species present in various liquid nuclear waste streams under different experimental conditions.

2. Natural Clays (NCs) for Sorption of Radionuclides

Because of their numerous properties (thermal stability, high surface area, porous structure, cation exchange capacity, swelling behavior, etc.), NCs have been widely studied and applied in wastewater treatment/decontamination and in environmental waste management [72–83]. For decades, bentonite clay has been used as a potential buffer material in nuclear waste management in underground repositories globally [84–92]. Being ubiquitous in nature and one of the cheapest sorbent materials, NCs have attracted greater attention for their potential application in the sorption of hazardous/toxic metal ions as well as radionuclides from contaminated and waste water under different experimental conditions. More specifically, NCs have been highly useful for cationic hazardous/radionuclides/heavy

metal removal due to the ion exchange between charge balancing cations (as alkali and alkaline earth metals) present in clays and cationic contaminant species (heavy metals, radionuclides, etc.) in targeted waste/solutions. The sorption behavior of various ions on bentonite clays can be determined by their specific surface area, chemical and mineralogical composition and cation exchange capacity [93–96]. This section encompasses the key and recent developments on the selection, characterization and application of potential NCs for the selective as well as effective removal of various radioactive metal cations and their surrogates under different physicochemical experimental conditions. The major outcomes/findings regarding the sorption of targeted metal cations under the optimized testing conditions have been highlighted and discussed throughout the selected studies.

Yu et al. investigated uranium (U) (VI) sorption on montmorillonite at different experimental conditions (pH, contact time, initial concentration of U(VI) solution, temperature and competing ions (NH₄⁺, K⁺, Mg²⁺, SO₄²⁻, CO₃²⁻, NO₃⁻ and HCO₃⁻)) via the batch method [97]. The chemical composition, the morphology before and after adsorption and the surface functional groups of montmorillonites were determined by x-ray fluorescence (XRF), scanning electron microscopy (SEM) and Fourier transform infrared spectroscopy (FT-IR) analyses. The experimental results for the effect of contact time on the sorption of U(VI) on montmorillonite exhibited fast sorption kinetics as the sorption equilibrium was acquired within 30 min. The sorption data revealed that the removal of U(VI) from montmorillonite was altered by pH, temperature and competing ions. U(VI) sorption was enhanced at a pH between 4.0 and 7.0 and then lowered with further increased pH values. Moreover, the presence of competing cations $(NH_4^+, K^+ \text{ and } Mg^{2+})$ in the testing solution impeded the sorption of U(VI) from montmorillonite, more specifically Mg²⁺, which decreased U(VI) sorption by approx. 40% (from approx. 70% to approx. 30%). The presence of CO_3^{2-} in the solution affected the aqueous speciation of U(VI) and formed strong uranyl carbonate complexes, which, in turn, influenced the sorption by increasing the solubility of uranium. The U(VI) sorption from montmorillonite displayed the applicability of the second-order kinetic model and the Langmuir isotherm model, where the determined maximum adsorption capacity was close to the experimental data. The authors also calculated the thermodynamic parameters (the changes in free energy (ΔG), enthalpy (Δ H) and entropy (Δ S)) at different temperatures (298 K, 308 K and 318 K) and the results indicated that the adsorption of U(VI) on montmorillonite was an endothermic and spontaneous process.

Tran et al. reported the sorption behavior of uranium and cesium on bentonite in carbonate-rich environments as a function of ionic strength (both in low ionic strength (2.20 mM) artificial rainwater (ARW) and high ionic strength (169 mM) ARW), colloid concentration and initial metal concentration [98]. The sorption experiments were performed using salt concentrations to mimic those measured in the northern Negev Desert, Israel, so that the obtained data could be applied to interpret the U(VI) and Cs mobility under site-specific geochemical conditions. The presence and absence of U(VI) in ARW altered the Cs sorption on bentonite significantly (Figure 1). The Cs sorption was decreased by approx. 10%, containing 1-2 g/L bentonite colloids in the presence of U(VI) in ARW. Moreover, the sorption of U(VI) was decreased compared to Cs on bentonite under similar experimental conditions, which was attributed to the formation of stable ternary calcium-uranyl carbonate compounds in the solution $(CaUO_2(CO_3)_2^{2-(aq)})$ and $Ca_2UO_2(CO_3)_3(aq))$, rather than their interaction with bentonite. These ternary calcium-uranyl carbonate complexes can control U(VI) migration through a carbonate rock aquifer. Based on the results, the authors concluded that in brackish carbonate rock aquifers, both U(VI) and Cs can migrate as dissolved species rather than as colloid-associated solids. Furthermore, bentonite colloids in brackish solutions similar to AGW can be aggregated and deposited, thereby hindering the mobilization of both U(VI) and Cs.



Figure 1. Competitive metals' sorption (in % and K_d)) in artificial rainwater (ARW) in a (**A**,**B**) high colloid concentration (0.1–2 g/L) and in a (**C**,**D**) low colloid concentration (0.01–0.5 g/L). Initial concentrations: $Cs = 7.5 \times 10^{-6}$ M, U(VI) = 3.6×10^{-6} M. Contact time: 14 days (adapted from reference [98]).

Philipp et al. very recently simulated the geochemical conditions (high pH and complex solution compositions) for radioactive waste in deep geological repositories and investigated the interactions of radionuclides with mineral surfaces under extreme conditions [99]. The authors reported the potential effect of Ca(II) on the retention capacity of Ca-bentonite for U(VI) and Np(VI) in hyper alkaline conditions using batch sorption experiments using time-resolved laser-induced luminescence spectroscopy (TRLFS). Zeta potential measurements as well as Ca²⁺ or Sr²⁺ sorption data on Ca-bentonite revealed that the alkaline earth metals sorbed efficiently between pH 8 and 13, which could be due to the partial compensation of the negative surface. Moreover, the uptake of U(VI) and Np(VI) in the absence and presence of Ca^{2+} or Sr^{2+} indicated that these cations considerably improved the radionuclide retention capacity of kaolinite and muscovite at pH \geq 10. Site-selective TRLFS analysis data confirmed the presence of two U(VI) species on the alumosilicate surfaces. First, a ternary U(VI) complex, in which U(VI) is bound to the bentonite surface via bridging Ca^{2+} with the surface $\equiv Ca - OH - U(VI)$). Second, U(VI) uptake in the interlayer of calcium (aluminum) silicate hydrates (C-(A-)S-H). Finally, the authors concluded that the presence of alkaline earth elements could trigger the high retention capacity of hexavalent actinides on bentonite under hyper-alkaline repository conditions and, therefore, can be significant for the long-term retention of U(VI) and Np(VI) in underground repositories under the extremely alkaline geochemical conditions.

Seliman et al. investigated the uptake of mono-, di- and trivalent radionuclides from the contaminated solution using bentonite clay via the batch method [100]. In this study, the authors tested the sorption behavior of four radionuclides $(^{134}Cs(I), ^{90}Sr(II),$ ¹³³Ba(II) and ¹⁵²Eu(III)) on Egyptian bentonite (Bent) and its Na⁺-modified form (Na-Bent) under different experimental conditions (contact time, pH of the solution and metal ion concentration) to identify the optimized parameters for their large scale application. The bentonite clay was obtained from the Alexandria governorate, Arab Republic of Egypt and was characterized by x-ray diffraction (XRD), XRF, SEM and ICP-OES techniques. The sorption of ¹³⁴Cs, ⁹⁰Sr, ¹³³Ba and ¹⁵²Eu on Bent and Na-Bent was observed to be highly dependent on the contact time, pH value and initial metal concentrations. The effect of contact time on the sorption of radionuclides on bentonite clay revealed fast sorption kinetics as the equilibrium was achieved within 10 min. The sorption data were applied well to the pseudo-second-order kinetic model. The distribution coefficient (K_d) values were calculated for the sorption of ¹³⁴Cs, ⁹⁰Sr, ¹³³Ba and ¹⁵²Eu o bentonite at different metal ion concentrations. The K_d values were determined in the order of $^{152}Eu > ^{90}Sr > ^{134}Cs > ^{133}Ba$ at lower metal ions' concentrations for their sorption on both sorbents (Bent and Na-Bent). However, the K_d of ¹³⁴Cs was found to be higher than ¹⁵²Eu after 150 ppm for Bent and after 200 ppm for Na-Bent. ⁹⁰Sr and ¹³⁴Cs were preferably sequestered onto Na-Bent rather than Bent, especially at higher concentrations. The ¹³⁴Cs, ¹³³Ba, ⁹⁰Sr and ¹⁵²Eu adsorption data on Bent and Na-Bent were fitted by non-linear equations of Langmuir and Freundlich isotherm models and the linear equation of the Dubinin-Radushkevich (D-R) model. The applicability of the Langmuir isotherm model was observed for all metals adsorption on Bent and Na-Bent except for ¹³³Ba the adsorption on Bent. The results for ¹³³Ba adsorption on Bent followed the D-R model, whereas the Freundlich isotherm model was applied for ¹⁵²Eu on Na-Bent. The adsorption energy values for the sorption of all metal ions were calculated from the D-R model and were found to be $>8 \text{ kJ mol}^{-1}$, which revealed that the sorption of these radionuclides on Bent and Na-Bent was mainly controlled by chemical reactions (by both cation exchange and surface complexation). Based on these results, the authors proposed that Egyptian bentonite with excellent adsorption capacity could be used as a potential sorbent for the treatment of aqueous waste as well as a buffer material in geological disposal sites.

Semenkova et al. reported Cs⁺ sorption on bentonite (obtained from the Kutch region of Gujarat, India) in the presence and absence of competing ions [101]. The major objective of the study was to develop a thermodynamic model to explore the possible interactions of Cs⁺ with different types of clay materials and to investigate the competition involving different cations in complex solutions. The obtained (raw) clay was crushed and sieved to size fractions $< 75 \mu m$, which the authors termed as original 'Kutch clay'. Later, the original clay sample was chemically treated and converted into Na-form. To compare the performance of Kutch clay, the authors also used a well-known FEBEX bentonite from Spain in its sodium form (Na- FEBEX). All the solid samples were characterized using XRD, FT-IR, XRF and BET analyses. The two-site exchange mathematical thermodynamic model for the sorption of Cs⁺ on raw Kutch clays in different experimental conditions was developed. The study compared Cs⁺ sorption on raw Kutch clay and FEBEX clay over a wide range of pH and concentrations (Figure 2). The experimental results were fitted in non-linear isotherms for Cs⁺ sorption on both clays and the shapes of the isotherms encouraged the use of a two-site exchange thermodynamic model. The presence of different cations (Ca²⁺, Mg²⁺ and H⁺) in the solution considerably altered the Cs⁺ sorption on applied clay sorbents.



Figure 2. Cs⁺ sorption on Na-form of FEBEX (**A**), purified (**B**) and original Kutch clays (**C**). Release of Al^{3+} from Na-form of FEBEX clay and purified Kutch clay (**D**) and Ca²⁺ and Mg²⁺ from original Kutch clay and Na-form of FEBEX clay (**E**). Clays concentration = 1 g/L, ionic strength (I) = 0.01 M (adapted from reference [101]).

Kim and Lee very recently simulated and interpreted the adsorption behavior of 22 elements/radionuclides (Am, Ac, Co, Cm, Cd, Cs, Cu, Na, Np, Ni, Nb, U, Sr, Sn, Pb, Pa, Pu, Po, I, Tc, Th and Zr) on bentonites using a machine-learning method [102]. The authors for the first time used the random forest (RF) method to predict the K_d values using a machine-learning model based on the Japan Atomic Energy Agency Sorption Database (JAEA-SDB). The study determined a database of ten input variables (pH, K_d , ionic strength, cation exchange capacity, radionuclide concentrations, solid–liquid ratio, oxidation number, surface area, ionic radius and electronegativity) for the RF model calculation. The study included two hyperparameters (the maximum number of variables to divide each node and the number of decision trees) together with the random seeds inside the RF model. The root mean square error values and correlation coefficient (R^2) values were used to create

the database for the K_d prediction and the performance of the RF on the defined systems. The R² values were observed to be 0.8604 and 0.9175 for the nested cross-validation method and the general RF model, respectively, which indicated the meaningful applicability of the machine-learning method in predicting the K_d values. Additionally, the authors suggested that, apart from the investigated 22 radionuclides, this approach could be significantly applied to predict the K_d values for other radionuclides as well (based on their ionic radius, oxidation number and electronegativity). Finally, the authors proposed that their approach could be highly applicable to developing the K_d values prediction model for the interaction of various radionuclides on bentonites via a machine-learning algorithm and to simulate their retention behaviors in the deep geological repositories.

Izosimova et al. investigated the sorption of Cs(I) and Sr(II) on bentonites from the simulated aqueous waste solutions of cesium and strontium nitrates with different compositions and under various pH (3, 7 and 10) conditions [103]. Bentonite samples were collected from the deposits of Taganskoe (sample 'T' from the Republic of Kazakhstan), Dash-Salakhlinskoe (sample 'DS' from the Republic of Azerbaijan), Zyryanskoe (samples 'Z' from the Kurgan region, Russia) and 10th Khutor ('10H' from the Republic of Khakassia). The solid samples were characterized via different techniques (XRD, FT-IR and zeta potential measurements). The aqueous suspensions of bentonite T and DS attained pH values over nine and exhibited the presence of a considerable amount of non-silicate iron compounds as 1.0% and 0.5%, respectively. The uptake capacity of the clay materials for Cs(I) and Sr(II) from aqueous solutions varied from 50% to 90% and was displayed in the following order: T > DS > Z > 10 H. The sorption mechanism of Cs(I) and Sr(II) on bentonites was interpreted to be an outer-sphere complex formation on planar surfaces due to the ion exchange reaction. Moreover, at <0.5 mol/L Cs(I) and Sr(II) concentrations in the initial solutions, the Cs⁺ and Sr²⁺ sorption on bentonite was accompanied by competitive interactions with protons at pH < 6 and with mono- and divalent cations (Na⁺, K⁺, Ca²⁺ and Mg^{2+}) at pH > 6.

Parisi reported the adsorption and separation behavior of cerium (III), lead (II) and crystal violet dye (CV) on K10-montmorillonite sorbent [104]. The selected montmorillonite clay was characterized by XRD and UV-visible spectroscopy. The study simulated the adsorption of Ce (III) and Pb (II) on montmorillonite mineral clay as the selective uptake of "model" contaminants from a ternary aqueous system. The authors tested various experimental conditions, including the solution pH, contact time, pollutant concentration and amount of montmorillonite clay. The obtained experimental data for the removal of all three contaminants on K10-montmorillonite were tested and analyzed via adsorption isotherms and the kinetic models. The results indicated considerable changes in the adsorption behavior of all three species under the defined experimental conditions. The Pb(II) and CV contaminants were adsorbed on K10-montmorillonite both in the acidic (pH 2) and near neutral (pH 6) conditions; however, the Ce(III) metal ions were adsorbed at pH 6 only, not in strongly acidic conditions (at pH 2). The authors proposed that the adsorption of CV dye on K10-montmorillonite attributed to the cation exchange as well as the surface complexation mechanism, involving the interlayers and surfaces/edges of montmorillonite. However, the removal of both metal cations (cerium (III) and lead (II)) from montmorillonite was achieved via cation exchange occurred at the clay surface. Finally, the authors concluded that the simultaneous decontamination of wastewaters containing multicomponent hazardous species could be achieved via rational experimental design and by optimizing the experimental conditions.

3. Modified/Functionalized Clays (MCs) for Sorption of Radionuclides

While NCs are widely used to sequester various cationic radionuclides, their modifications can further improve the removal capacity of hazardous contaminants from waste solutions [105–109]. The modification of NCs using organic cations (hexadecyl trimethyl ammonium (HDTMA), Benzethonium (BE) and hexadecyl pyridinium (HDPy), etc.) and inorganic additives can considerably alter their removal capacity of different anionic radionuclides [110–113]. During modification, the organic cations preferentially occupy negatively charged sites on the clay surface, changing the net charge on the minerals, which eventually triggers enhanced removal of anionic species by the ion exchange mechanism [114–117]. In this section, we discuss some key and recent contributions made toward the modification of NCs using different organic/inorganic surfactants/additives and their application for the removal of various radionuclide species from waste solutions under different experimental conditions.

Yang et al. very recently studied ⁹⁹Tc removal from radioactive wastewater using modified bentonite using batch and column experiments [118]. The authors synthesized hexadecylpyridinium-chloride-monohydrate-modified bentonite (HDPy-bent) and applied it for the removal of perrhenate ReO_4^- (an analog for radioactive pertechnetate ($^{99}\text{TcO}_4^-$)) under different testing conditions, such as the solution pH, contact time, initial concentration and competing anions. The solid material was characterized by FT-IR, x-ray photoelectron spectroscopy (XPS) and transmission electron microscopy combined with an energy-dispersive x-ray spectrometer (TEM-EDS). The obtained results revealed that the HDPy-bent sorbent exhibited outstanding ReO₄⁻ uptake capacity at a wide range of pH (2–10), fast adsorption kinetic and good adsorption capacity. Figure 3A–F exhibit the effects of pH and ionic strength on Re(VII) sorption on HDPy-bent, the zeta potential data of HDPy-bent sorbent prior to and after Re(VII) sorption, pseudo-second adsorption kinetic models at different temperatures and the UV-vis analysis results of Re(VII) sorption on HDPy-bent. The modified bentonite sorbent displayed high selectivity toward ReO_4^- in the presence of various competing anions (PO_4^{3-} , SO_4^{2-} , CO_3^{2-} , Cl^- , NO_3^- and HCO_3^-) as well as humic acid (HA). The uptake mechanism was interpreted as an anion exchange between ReO₄⁻ and Cl⁻ and a precipitation reaction between HDPy⁺ and ReO4⁻ in low concentration and high concentration, respectively. Moreover, based on the density functional theory (DFT) calculation, the authors suggested that ReO_4^- was adsorbed on the top of the pyridinium ring of HDPy⁺, establishing a face-to-face stacking with pyridinium ring and p- π interaction between O (of ReO₄⁻) and pyridinium ring (Figure 4). The excellent selectivity of HDPy-bent toward ReO_4^- was attributed to the sturdier p- π interaction in HDPy⁺/ReO₄⁻ compared to the same interaction in HDPy⁺/other anions. Desorption experiments indicated that the HDPy-bent exhibited good durability and reusability. Finally, the authors concluded that their study demonstrates a low-cost adsorbent, which can potentially be used for the efficient removal of ⁹⁹Tc from wastewater.

Very recently, Hong et al. investigated the application of surface-modified bentonite for the removal of Cs and Sr from (simulated) groundwater conditions [119]. The study proposed that the surface-modified bentonite could enhance the safety of radioactive waste in deep geological repositories by improving the radionuclide sorption capacity of bentonite. The authors applied an inorganic synthesis route to modify the bentonite surface to form NaP zeolite and to improve the radionuclides' (Cs and Sr) removal efficiency on contaminated groundwater. They conducted a surface modification of bentonite (obtained from Yeonil, South Korea) by adding inorganic additives to form NaP zeolite on the surface of the bentonite. Sodium aluminate, sodium metasilicate and sodium hydroxide chemicals were used to synthesize NaP zeolite phases on the bentonite surface at 95 °C. XRD patterns and SEM analysis data confirmed the preparation of NaP/bentonite materials with spherical clusters of NaP zeolite covered on the surface of bentonite. FT-IR spectra and the Barrett-Joyner-Halenda (BJH) pore size distribution (PSD) analyses revealed that the water content and the macropores of NaP/bentonite composite materials were lowered compared to that of the original bentonite, which resulted in a smaller specific surface area of the NaP/bentonite composite compared to the original bentonite. The NaP-zeolite-modified bentonite surface enhanced the sorption efficiency of Cs and Sr. The original bentonite exhibited 72.8% and 70.5% of removal efficiencies of Cs and Sr, respectively, whereas, 10 wt% NaP/bentonite sorbed 95.4% Cs and 99.1% of Sr under the rest of the similar experimental conditions. Based on the results, the authors suggested that their approach



could be significant for a facile surface modification and the application of bentonite for the efficient removal of radionuclides.

Figure 3. The effects of ionic strength and pH on Re(VII) adsorption on HDPy-bent (**A**). The zeta potential measurement of HDPy-bent prior to and after Re(VII) adsorption (**B**). Pseudo-second adsorption kinetic models fitting at 25 ± 1 °C (**C**), 40 ± 2 °C (**E**) and 55 ± 2 °C (**F**) and UV–vis spectra of Re(VII) adsorption on HDPy-bent (**D**) (adapted from reference [118]).

HDPv⁺

LUMO



Figure 4. Molecular orbital diagrams of $HDPy^+/ReO_4^-$ and $HDPy^+/NO_3^-$ presenting face-to-face stacking in HDPy⁺/ReO₄⁻ and side-to-side stacking in HDPy⁺/NO₃⁻ (adapted from reference [118]).

 $p-\pi$ interaction

π

 $p-\pi$ interaction

Soliman et al. applied thoron (TH, C₁₆H₁₁AsN₂Na₂O₁₀S₂)) modified montmorillonite for the sorption of cobalt radionuclide (Co(II)) from aqueous waste solutions [120]. The authors used FT-IR and XRD for qualitative analysis of the surface functional groups of the adsorbate and to confirm the organo-modification of the surface-modified montmorillonite. The characterization data revealed that TH was attached at the external sites of TH-modified montmorillonite (TMM) rather than the interlayer space. The authors performed batch adsorption experiments under various experimental conditions (contact time, pH, ionic strength, adsorbent dosage, adsorbate concentration and temperature). The obtained results suggested that the sorption of Co(II) was largely reliant on the pH as well as the quantity of the modifying agent loaded onto the montmorillonite. TMM achieved an enhanced Co(II) removal efficiency compared to its unmodified montmorillonite counterpart, which were calculated as 99% and 63%, respectively. Kinetic studies revealed 41.2%, 69.9% and 99.7% Co(II) sorption in 30, 20 and 10 min using 0.4, 0.2 and 0.1 mmol/L initial cobalt(II) concentration in solutions, respectively. The obtained results demonstrated the applicability of the pseudo-second-order kinetic model in the applied experimental conditions. Ionic strength played a significant role in the removal of Co(II) from waste solution, as Co(II) sorption was found to be 99% and 40% at 0.0001 and 0.1 M NaCl, respectively. The authors observed the applicability of both Langmuir and Freundlich isotherm models for Co(II) removal from aqueous waste. In the final experiments, the authors varied the reaction temperature to 30 °C, 45 °C, and 60 °C and calculated the thermodynamic parameters (changes in Gibbs free energy (ΔG) and enthalpy (ΔH)) for sorption of Co(II) on TMM sorbent. The results suggested that the cobalt(II) sorption on modified montmorillonite was spontaneous and endothermic in nature ($\Delta G = -2.687$, -3.407 and -4.127 kJ/mol at 30 °C, 45 °C and 60 °C, respectively; $\Delta H^{\circ} = 11.857 \text{ kJ/mol}$). Based on the results, the authors claimed that organo-modified montmorillonite (TMM) can be potentiality applied as an

effective sorbent for the removal of Co(II) and other cationic radionuclides from aqueous radioactive waste under the optimized testing conditions.

Hu and Tan modified attapulgite clay for the sorption of Th(IV) ions from an aqueous solution at different contact times, sorbent contents, pH levels, ionic strengths and in the presence of fulvic acid (FA)/humic acid (HA) [121]. The authors investigated the sorption behavior and interaction mechanism of Th(IV) ions on the organo-modified attapulgite (OA). The study used ammonium citrate tribasic to modify the attapulgite clay and the OA was characterized by XRD and FT-IR techniques. The obtained sorption results suggested that the ammonium-citrate-tribasic-modified attapulgite clay exhibited an efficient Th(IV) sorption ability on aqueous waste solutions. Kinetic experiments revealed fast Th(IV) sorption on OA and the obtained data followed a pseudo-second-order kinetic model. Moreover, the Th(IV) sorption on OA was largely dependent on the pH, ionic strength and Th(IV) initial concentration. In the presence of HA/FA, the sorption of Th(IV) on OA sorbent was enhanced at pH < 4, whereas there was no considerable change in the sorption at pH > 4. Moreover, Th(IV) uptake on modified attapulgite was determined to be an endothermic process. The authors also varied the initial Th(IV) concentration in the sorption solution and tried to analyze the experimental data using three adsorption models (Langmuir isotherm, Freundlich isotherm and D-R isotherm). Among these, the Th(IV) adsorption data applied well in the Langmuir isotherm model compared to the Freundlich and D-R isotherm models. Finally, the authors concluded that the high Th(IV) sorption capacity of OA at fast reaction kinetics could make modified attapulgite a potential candidate for the removal of Th(IV) from waste solutions.

Ding et al. applied nickel MC for the Cs sorption on aqueous waste solutions [122]. The authors investigated the kinetics and sorption capacity of the pristine and modified akadama clay (AC) under various experimental conditions (solution pH, contact time, adsorbent dosage, Cs^+ concentration, competitive cations and temperature). The study used lake water for the simulation of low-strength Cs⁺-contaminated water. Cs⁺ contamination in the lake in Japan is a typical water pollution (due to stormwater) related to radioactive accidents, and could result from the nuclear power stations due to some unavoidable natural disasters. The lake water was collected from the Matsumi Lake of the University of Tsukuba, Japan, which mainly receives stormwater. Prior to the experiments, the lake water was filtrated with a glass microfiber. Ni-modified AC clay (solid) samples were characterized via various techniques: XRD, FT-IR, N₂ adsorption–desorption isotherms, SEM, EDS and thermo-gravimetric/differential thermal analyzer (TG/DTA). After modification, the AC was transformed into a typical mesoporous material and the pore diameter of the Ni-modified AC was reduced from >20 nm to <12 nm. Over 90% Cs⁺ removal from the modified AC sorbent was achieved from pH > 11 to pH \geq 5 (alkaline to neutral conditions). The adsorption capacity of the Ni-modified AC was found to be 16.1 ± 0.9 mgg⁻¹ (2.5 times higher than the pristine AC). The presence of competitive ions $(Na^+ and K^+)$ in the waste solution resulted in reduced Cs⁺ adsorption, especially in the presence of K^+ , which was attributed to the similar characteristics of K^+ and Cs^+ (Figure 5). Moreover, 85% Cs⁺ removal efficiency on modified AC was achieved using lake water samples. Based on the results, the authors concluded that the modified AC was efficiently applied for Cs⁺ removal from low-strength Cs⁺-contaminated water and could be a suitable option analogous to other clays with respect to Cs⁺ adsorption capacity and selectivity.

Shakir et al. reported the simultaneous removal of monovalent (¹³⁷Cs), divalent (⁶⁰Co) and trivalent (^{152 + 154}Eu) radionuclides, as well as chromotrope 2B (an anionic pollutant; C2B) from mixed radioactive process wastewater (MRPWW) using organo-bentonite (bentonite partially modified (PMB) with the cationic surfactant cetyltrimethylammonium bromide (CTAB)) [123]. The authors performed batch kinetics and isotherm studies to evaluate the adsorption efficiency of PMB. After modification, the organo-bentonite was characterized by XRD, XRF spectrophotometer and infrared spectroscopy. The modified PMB (78% of the cation exchange capacity) adsorbed all radionuclides and C2B from aqueous waste solutions. The study followed the pseudo-first-order kinetics and Langmuir

adsorption isotherm model, which further indicated the sorption was mainly governed by intra-particle diffusion and external surface adsorption. Electrostatic and hydrophobic interactions were proposed to be the major sorption mechanism between C2B and PMB, whereas ion exchange appeared to be the main sorption mechanism for the removal of radionuclides from modified bentonite. The modified bentonite was capable of approx. 100% sorption of C2B and all radionuclides from the simulated MRPWW. The removal efficiency of PMB for ¹³⁷Cs(I), ⁶⁰Co(II) and ^{152 + 154}Eu(III) were found to be 0.732 mmol/g, 0.644 mmol/g and 0.555 mmol/g, respectively. The thermodynamic parameters' values (changes in free energy and enthalpy) at different sorption temperatures suggested that the sorption of C2B and radionuclides on PMB followed exothermic and spontaneous sorption.



Figure 5. Effect of competing ions (Na⁺ and K⁺) on the removal efficiency (**A**,**C**,**E**) and distribution coefficient (**B**,**D**,**F**) of Cs⁺ sorption on the pristine and modified akadama clay (AC). Initial Cs⁺ concentration: 1 ppm, dosage: 2.5 g L^{-1} (adapted from reference [122]).

Guerra et al. modified diquite (D) and bentonite (B) clay minerals (obtained from the Amazon region, Brazil) using 5-mercapto-1-methyltetrazole (MTTZ) and investigated the application of MCs for the adsorption of Th(IV) under different experimental conditions (solution pH, contact time and variation of Th(IV) concentration) [124]. After chemical treatment, the MC (DMTTZ and BMTTZ) materials were characterized by XRD, N2 adsorption/desorption isotherms, TEM and ¹³C NMR spectroscopy. The BET surface areas (SBET) of the chemically modified materials changed significantly due to the creation of additional microspores in the solid structure. The BET surface areas of the BMTTZ and DMTTZ samples were determined as $398.5 \text{ m}^2 \text{ g}^{-1}$ and $178.8 \text{ m}^2 \text{ g}^{-1}$, respectively, which were found to be quite higher than their natural counterparts, as the SBET of B and D samples were calculated as 41.42 m² g⁻¹ and 5.0 m² g⁻¹, respectively. The authors used a calorimetric titration procedure to determine the energetic effect (changes in enthalpy, free energy and entropy; ΔH° , ΔG° and ΔS°) at the solid–liquid interface caused by Th(IV) and phyllosilicate surfaces' interaction and predicted that Th(IV) sorption on DMTTZ and BMTTZ sorbents were spontaneous and entropy-favorable. The maximum number of moles of Th(IV) adsorbed on DMTTZ and BMTTZ were determined as 10.45×10^{-2} mmol g^{-1} and 12.76 \times 10^{-2} mmol g^{-1} , respectively.

In another study, Guerra et al. reported the application of raw and chemically modified hectorite clays (obtained from the Amazon region, Brazil) for Th⁴⁺, U⁶⁺ and Eu³⁺ uptake from aqueous solutions [125]. Hectorite (H) clay was modified with 2-mercaptobenzimidazole (MBI) via heterogeneous and homogeneous treatment methods. The clay samples (both natural and modified materials) were analyzed using SEM and magic-angle-spinning nuclear magnetic resonance (MAS ²⁹Si and ¹³C NMR) spectroscopy. After chemical modification, the peaks in the 112–129 ppm region in the ¹³C NMR spectra confirmed the attachment of organic functional groups onto the hectorite clay. The adsorption of Th⁴⁺, U⁶⁺ and Eu³⁺ on MC from aqueous waste solution followed the order of Eu³⁺ > U⁶⁺ > Th^{4+.} The maximum number of moles adsorbed on modified hectorite clay was calculated as 14.01 mmolg⁻¹, 12.85 mmolg⁻¹ and 11.63 mmolg⁻¹ for Eu³⁺, U⁶⁺ and Th⁴⁺ metal ions, respectively.

Makarov et al. investigated the application of activated carbon (additives) for the modification of bentonite and the role of composite/modified material in the removal of pertechnetate ions (TcO₄⁻) from waste solution and its impact on the immobilization in bentonite-based engineered barriers [126]. The activated-carbon-modified bentonite clay composite materials were obtained by adding bentonite clays with 0.5, 1 and 5% mass ratios of milled activated carbon/graphite AG-3 (activated graphite derived from low-temperature coke dust/charcoal) and CAU (activated carbon of coconut shell). Synthesized sorbents were analyzed using XPS and X-ray absorption near-edge spectroscopy (XANES)/extended x-ray absorption fine structure (EXAFS) spectroscopy. The sorption tests were performed at a solid-to-liquid ratio of 1:20 in oxidizing conditions. The obtained sorption results suggested that the activated-carbonmodified bentonite clay (0.5–5% activated carbon (by weight) in the bentonite clay) significantly enhanced the TcO₄⁻ sorption capacity and achieved the K_d value up to 740 mLg⁻¹. Finally, the authors suggested that the activated carbons (AG-3 and CAU) could be promising components for engineered clay-based barriers to mitigate the ⁹⁹Tc mobility in the environment under oxidizing conditions.

4. Conclusions and Future Perspectives

In this review, we have highlighted and discussed some of the key as well as recent progress toward the design, characterization and applications of various clays for efficient sorption of both cationic and anionic radioactive contaminants from simulated waste solutions under different experimental conditions. We have particularly demonstrated the application of NCs (bentonite/montmorillonite) for the removal of different metal ions (surrogates of radionuclides) in the optimized experimental conditions. Moreover, this review has equally focused on the functionalization and modification of clay materials to improve the uptake capacity of the various radionuclides possessing variable chemical properties

and high mobility behavior in the environment. Prior to the sorption experiments, the applied clays (both natural and functionalized clays) were efficiently characterized using various techniques (XRD, SEM, FT-IR, XPS, N2 adsorption/desorption isotherms, TGA, etc.) to analyze the key properties and their influences on the removal capacities of targeted ions/radionuclides. More specifically, the detailed characterization of modified clays with organic moieties has been widely discussed. The sorption efficiencies of various clays have been studied under different physicochemical conditions (pH, contact time, ionic strength, temperature, sorbent and sorbate doses, etc.) and these materials have displayed the potential sequestration of both anionic and cationic hazardous species at the optimized testing conditions (Table 1). The obtained results suggested that NCs have been very effective for the removal of metal cations, whereas modified/functionalized clay materials have exhibited excellent sorption capacities for both cationic and anionic specifies. The key aspects of this behavior of NCs and MCs can be attributed to their net/overall charges in aqueous media. It is noteworthy that NCs possess an overall negative charge in aqueous media and, therefore, are most suitable for the sequestration of metal cations/radionuclides. However, after functionalization/modification, the MCs can also be suitable for the efficient removal of anionic radionuclides/hazardous anions, as discussed and highlighted here. Therefore, being abundant and ubiquitous in nature, the clays can be effectively applied for the treatments of various radioactive contaminants (cations as well as anions) from waste solutions.

Table 1. Sorption of radionuclides from (simulated) waste solutions using natural clays (NCs) and modified clays (MCs) at various experimental conditions.

Types of Materials	Used Clays and Radionuclides /Metal Ions	Experimental Conditions, Characterization Techniques and Sorption Performances	References
	Montmorillonite U(VI)	U(VI) = 30 ppm; pH = 4.0–7.0; Ionic strength (I) = ND; contact time = 30 min (m); temperature = 25–45 °C; sorbent concentration: ND. Characterization techniques: XRF, SEM and FT-IR spectroscopy. Performance: 72% U(VI) removal; 18.13 mgg ⁻¹ maximum adsorption capacity.	Yu et al. [97]
	Bentonite U(VI) and Cs(I)	$\begin{split} U(VI) &= 3.7 \times 10^{-6} \text{ M}, \text{Cs}(I) = 7.5 \times 10^{-6} \text{ M}; \text{pH} = 7.0-8.0;\\ I &= \text{ND}; \text{ contact time} = 14 \text{ days} (d); \text{ temperature} = 25 \ ^\circ\text{C};\\ \text{ sorbent concentration}: 0.1-2.0 \text{ g/L}.\\ \text{ Characterization techniques: XRD and zeta}\\ \text{ potential measurement.}\\ \text{Performance: approx.5\% U(VI) removal; 31\% Cs(I) removal.} \end{split}$	Tran et al. [98]
NCs	Bentonite and kaolinite U(VI) and Np(VI)	$\begin{split} U(VI) &= 5 \times 10^{-7} \text{ M}, \text{Np}(VI) = 1 \times 10^{-7} \text{ M}; \text{pH} = 8.0 - 13.0; \text{ I} = 0.1 \\ \text{M; contact time} &= 7 \text{ d for } U(VI) \text{ and } 3 \text{ d for } \text{Np}(VI); \\ \text{temperature} &= 25 ^{\circ}\text{C}; \text{ sorbent concentration: } 2.0 - 10.0 \text{ g/L}. \\ \text{Characterization techniques: TRLFS and zeta} \\ \text{potential measurement.} \\ \text{Performance: } >90\% \text{ U}(VI) \text{ and } \text{Np}(VI) \text{removal.} \end{split}$	Philipp et al. [99]
	Bentonite Cs(I), Sr(II), Ba(II) and Eu(III)	Cs(I), Sr(II), Ba(II) and Eu(III) = 10 ppm; pH = 2–12; I = 0.01 M; contact time = 20 m; temperature = 25 °C; sorbent concentration = 10 g/L. Characterization techniques: XRD, XRF, SEM and ICP-OES. Performance: = 87% Cs(I) removal, 98% Sr(II) removal, 100% Ba(II) and Eu(III) removal.	Seliman et al. [100]
	Bentonite Cs(I)	$Cs(I) = 10^{-11}-10^{-2}$ M; pH = 2-11; I = 0.01 M; contact time = 1-300 h (h); sorbent concentration = 1 g/L. Characterization techniques: XRD, FT-IR, XRF and BET analysis. Performance: approx. 100% Cs(I) removal.	Semenko-va et al. [101]
	Bentonite Cs(I) and Sr(II)	$ \begin{array}{l} Cs(I), Sr(II) = 2.5 \times 10^{-5} - 5 \times 10^{-3} \text{ M}; \text{ pH} = 2.0 - 10.00; \\ I = 0.01 \text{ M}; \text{ contact time} = \text{ND}; \text{ sorbent concentration} = 10.0 \text{ g/L}. \\ Characterization techniques: XRD, FT-IR and zeta \\ \text{ potential measurement.} \\ Performance: 90-100\% \text{ Cs(I) and Sr(II) removal.} \end{array} $	Izosimova et al. [103]

Types of Materials	Used Clays and Radionuclides /Metal Ions	Experimental Conditions, Characterization Techniques and Sorption Performances	References
	K10-montmorill-onite Ce (III) and Pb (II)	Ce (III) and Pb (II)= 2–300 ppm; pH = 2 and 6; I = ND; contact time = 2–4 h; sorbent concentration = 0.4 g/L. Characterization techniques: XRD and UV-visible spectroscopy. Performance: approx. 100% Ce (III) and Pb (II) removal.	Parisi [104]
MCs	HDPy-modified bentonite ReO4 ⁻	$\label{eq:ReVII} \begin{array}{l} \text{Re(VII)} = \text{ND; pH} = 2.011.0; \text{I} = 0.0010.1 \text{ M; contact time} = \\ 1240 \text{ m; temperature} = 2555 \ ^\circ\text{C; sorbent concentration} = \text{ND}.\\ \text{Characterization techniques: FT-IR, XPS and TEM-EDS.}\\ \text{Performance: approx. } 100\% \ \text{ReO_4}^- \text{ removal.} \end{array}$	Yang et al. [118]
	Thoron-modified montmorillonite Co(II)	$ \begin{array}{l} \text{Co(II)} = 1 \times 10^{-4} - 4 \times 10^{-4} \text{ M}; \text{ pH} = 2.5 - 8.0; \text{ I} = 0.0001 - 0.1 \text{ M}; \\ \text{contact time} = 0 - 120 \text{ m}; \text{ temperature} = 30 - 60 \ ^\circ\text{C}; \text{ sorbent} \\ \text{concentration} = 10.0 \text{ g/L}. \\ \text{Characterization techniques: XRD and FT-IR spectroscopy.} \\ \text{Performance: approx. 100% Co(II) removal.} \end{array} $	Soliman et al. [120]
	Ammonium-citrate tribasic-modified attapulgite clay Th (IV)	Th(IV) = 5–15 ppm; pH = 1.5–11.5; I = 0.001–0.1 M; contact time = 24 h; temperature = 25 °C; sorbent concentration = 0.3 g/L. Characterization techniques: XRD and FT-IR analyses. Performance: approx. 100% Th(IV) removal.	Hu and Tan [121]
	Nickel (Ni) modified Akadama clay (AC) Cs(I)	eq:cs(I) = 10 ppm; pH = 2.0-12.0; I = ND; contact time = 0-24 h; temperature = 15-35 °C; sorbent concentration = 2.5-10 g/L. Characterization techniques: XRD, FT-IR, N2 ads-des isotherms, SEM, EDS and TG/DTA. Performance: >90% Cs(I) removal.	Ding et al. [122]
	Cetyltrimethylammonium- bromide (CTAB) modified bentonite (¹³⁷ Cs), (⁶⁰ Co) and (^{152 + 154} Eu)	$(^{137}Cs), (^{60}Co) and (^{152 + 154}Eu) = 1 \times 10^{-6} M; pH = 1.2-12.7;$ I = ND; contact time = 0–4 h; temperature = 30–50 °C; sorbent concentration = 10.0 g/L. Characterization techniques: XRD, XRF and IR spectroscopy. Performance: approx. 100% (^{137}Cs), (^{60}Co) and $(^{152 + 154}Eu)$ removal.	Shakir et al. [123]
	5-mercapto-1-methyltetrazole- modified diquite and bentonite Th(IV)	$\begin{array}{l} \text{Th}(\text{IV}) = 1.20 - 2.0 \times 10^{-5} \text{ M}; \text{ pH} = 1.0 - 8.0; \text{ I} = \text{ND};\\ \text{contact time} = 24 \text{ h}; \text{ temperature} = 25 \ ^\circ\text{C};\\ \text{sorbent concentration} = 1.0 \text{g/L}.\\ \text{Characterization techniques: XRD, N}_2 \ \text{ads/des isotherms, TEM}\\ \text{and} \ ^{13}\text{C} \ \text{NMR spectroscopy.}\\ \text{Performance: } 10.45 \times 10^{-2} \ \text{and} \ 12.76 \times 10^{-2} \ \text{mmol/g Th}(\text{IV})\\ \text{adsorbed onto } D_{\text{MTTZ}} \ \text{and} \ B_{\text{MTTZ}}, \ \text{respectively.} \end{array}$	Guerra et al. [124]
	2-mercaptobenzi-midazole- modified hectorite clay Th ⁴⁺ , U ⁶⁺ and Eu ³⁺	Th ⁴⁺ , U ⁶⁺ and Eu ³⁺ = $1.25-2.5 \times 10^{-5}$ M; pH = $1.0-8.0$; I = ND; contact time = 12 h; temperature = $25-40$ °C; sorbent concentration = 1.0 g/L. Characterization techniques: SEM, MAS ²⁹ Si and ¹³ C NMR spectroscopy. Performance: 11.63 mmol/g, 12.85 mmol/g and 14.01 mmol/g removal of Th ⁴⁺ , U ⁶⁺ and Eu ³⁺ , respectively.	Guerra et al. [125]
	Activated-carbon-modified	TcO ₄ ⁻ = 4×10^{-6} M; pH = ND; I = ND; contact time = 14 d; temperature = 25 °C; sorbent concentration = 1/40 (solid to	

Table 1. Cont.

ND refers to the not defined values.

bentonite,

TcO4

Despite the significant developments in the application of clays as effective sorbent materials for the sorption of various cationic and anionic species from contaminated wastewater [127–132], a few challenges are required to be addressed to improve their application in future studies. It is noteworthy that after post-functionalization, the modified clays have exhibited efficient removal of various radionuclides' species. However, clay modification using organics also has drawbacks in terms of cost efficiency, stability, environmental aspects and physicochemical characteristics of the modified organoclays on large-scale applications. Additionally, more detailed analytical approaches will be necessary to assess the overall performance of the functionalized clays for the treatment of radioactive waste in underground repositories. Future studies should focus on the application of theoreti-

liquid ratio, S/L).

Characterization techniques: XPS, XANES and EXAFS.

Performance: approx. 94% TcO₄⁻ removal.

Makarov et al. [126]

cal and computational approaches (density functional theory calculations and molecular dynamics simulations) to assess further insights into the interaction mechanism of metal contaminants–radionuclides with various clays. A combined advancement (in experimental, analytical and theoretical approaches) to define the overall physicochemical properties of clays and clay-based sorbents under optimized conditions can be significant for their broader application in (a) the efficient and selective sorption of radioactive contaminants from waste solutions, (b) retarding their mobility in the environment and, finally, (c) in nuclear waste management. In addition, the management of the resulting waste after the sorption of radionuclides on clays and the fate of the final disposal via the immobilization of generated waste (or various waste form formulation) can be found in Singh et al. [27].

Overall, future studies should focus on resolving the aforementioned challenges and enhancing the application of clays/clay-based materials for the selective and effective removal of radionuclides from waste solutions. To meet these challenges, a detailed comprehensive database on the characterization and interaction behavior of natural and functionalized clays with various radioactive species (both cations and anions) using advanced analytical tools and theoretical as well as computational approaches are necessary. Additionally, the economic and environmental aspects should be equally considered during the selection, design and application of clay-based sorbents for treatments of radioactive waste in further studies.

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